

# A generalized correlation for two-phase forced flow heat transfer—second assessment

V. V. KLIMENKO†

Cryogenics Department, Moscow Power Engineering Institute, 105835, Moscow, U.S.S.R.

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**Abstract**—A generalized correlation for two-phase forced flow heat transfer (nucleate boiling and vaporization) is suggested which is valid for vertical and horizontal channels with a fully wetted perimeter. It represents, with a mean absolute deviation equal to 14.4%, experimental data for 21 different liquids (water, organic liquids, freons and cryogenics) in the following ranges of the main parameters: pressure, 0.61–196 bar; heat flux density,  $10\text{--}8 \times 10^6 \text{ W m}^{-2}$ ; mass flow rate, 5.6–6240  $\text{kg m}^{-2} \text{ s}^{-1}$ ; channel diameter, 0.47–74.7 mm. It has been found that both the nucleate boiling and the vaporization heat transfer intensity depend on the thermal conductivity of the channel wall material. It is demonstrated that nucleate boiling heat transfer is described by a single equation in dimensionless variables containing an individual constant for each of the four groups of fluids (water, organic liquids, freons and cryogenics). Vaporization heat transfer is described by a single equation with a constant universal for all the fluids.

## INTRODUCTION

DURING the last decade, a number of papers [1–4] were published by the present author, where, on the basis of extensive information accumulated by that time on boiling and vaporization heat transfer in two-phase flow of various fluids, an attempt was made to suggest a universal predictive correlation. In a recent paper [5], a correlation for forced convection vaporization was obtained, which is valid for various fluids (water, freons, cryoagents) in a wide range of the main parameters specifying the process, as well as the relationship for nucleate boiling. But since the latter relation had been initially developed exclusively for cryogenic fluids, it was not subjected to intensive verification by experimental data for other substances. The aim of the present paper is to refine this relationship and to extend it to non-cryogenic fluids. This reassessment is based on extensive experimental information accumulated over the last four decades and, which is of no less importance, on the up-to-date data for thermal properties the acquisition and processing of which are being constantly carried out at the author's laboratory. Simultaneously the correlation for forced convection vaporization [5] was also subjected to verification against an expanded body of data.

## PREVIOUS STUDIES

In a recent paper [5], the following correlation has been suggested for two-phase flow heat transfer:

$$Nu_{TP} = \begin{cases} Nu_b & \text{with } N_{CB} < 1.6 \times 10^4 \\ Nu_c & \text{with } N_{CB} > 1.6 \times 10^4 \end{cases} \quad (1)$$

where the nucleate boiling heat transfer is found from the formula

$$Nu_b = 7.4 \times 10^{-3} Pe_*^{0.6} K_p^{0.5} Pr_1^{-1/3} K_\lambda^{0.15} \quad (2)$$

and the forced convection vaporization heat transfer is given by the relation

$$Nu_c = 0.087 Re_m^{0.6} Pr_1^{1/6} (\rho_v/\rho_l)^{0.2} K_\lambda^{0.09} \quad (3)$$

Equation (3) was verified by experimental data on evaporation of nine different liquids (water, freons, cryogenics) in wide ranges of the main governing parameters: pressure (0.61–30.4 bar), mass flow rate (50–2690  $\text{kg m}^{-2} \text{ s}^{-1}$ ), quality (0.017–1.00), channel diameter (1.63–41.3 mm). Altogether, 553 experimental points were used for comparison, with the mean absolute deviation from the calculation amounting to 12.9% against 24.1 and 34.2% for the familiar relationships of Chen and Shah, respectively.

Equation (2) was derived on the basis of the data for nucleate boiling of only cryogenic liquids (helium, hydrogen, neon, nitrogen and argon). Originally, 440 experimental points were used for processing, of which 417 points (i.e. almost 95%) deviated from those calculated by no more than 35%; the mean absolute deviation did not exceed 17%. Equation (2) was compared with selected data on boiling water and freons [8–12] showing a satisfactory agreement. However, on the whole, the question about the possibility of applying equation (2) to nucleate boiling heat transfer calculation for non-cryogenic liquids remains unsolved, because up to now this equation has not been thoroughly checked on a great number of experimental data.

## EXPERIMENTAL DATA USED TO DEVELOP AND TEST A NEW CORRELATION

Table 1 contains the principal information concerning experimental investigations that provided a

† Present address: Global Energy Problems Laboratory, Nuclear Safety Institute, 113191, Moscow, U.S.S.R.

## NOMENCLATURE

$a$	thermal diffusivity	Greek symbols	
$A$	Philippov's number, $100\pi$ at $\tau = 0.625$	$\alpha$	heat transfer coefficient
$b$	Laplace constant, $\sqrt{(\sigma/g(\rho_l - \rho_v))}$	$\theta$	contact wetting angle
$C_p$	specific heat per unit mass at constant pressure	$\lambda$	thermal conductivity
$d$	channel diameter	$\mu$	dynamic viscosity
$g$	body force acceleration	$\nu$	kinematic viscosity
$G$	total mass flow rate	$\pi$	reduced pressure, $P/P_{cr}$
$K_\lambda$	relative thermal conductivity, $\lambda_w/\lambda_l$	$\rho$	density
$K_p$	dimensionless parameter, $pb/\sigma$	$\sigma$	surface tension
$l$	channel length	$\tau$	reduced temperature, $T/T_{cr}$
$N_{CB}$	convective boiling number, $(rG/q)[1 + x(\rho_l/\rho_v - 1)](\rho_v/\rho_l)^{1/3}$	Subscripts	
$Nu$	Nusselt number, $\alpha b/\lambda_l$	b	boiling
$p$	pressure	c	two-phase forced convection (vaporization)
$Pe_*$	modified Peclet number, $qb/r\rho_v a_l$	calc	calculated
$Pr$	Prandtl number	cr	critical
$q$	heat flux density	e	equivalent
$r$	latent heat of vaporization	exp	experimental
$R_s$	mean microroughness	l	liquid at saturation
$Re_m$	Reynolds number of the mixture, $W_m b/\nu_l$	m	mixture
$T$	temperature	s	saturation
$\Delta T$	temperature difference, $T_w - T_s$	TP	two-phase
$w_m$	two-phase mixture velocity, $(G/\rho_l)[1 + x(\rho_l/\rho_v - 1)]$	v	vapour at saturation
$x$	quality.	w	wall.

basis for developing a new correlation. Use was made of nucleate boiling data for 21 various fluids (water, organic fluids, freons, cryogens) in vertical and horizontal pipes and annular channels (without stratified flow regimes). It should be pointed out that the initial file of experimental points (about 3000 points) was thoroughly analysed and after preliminary calculations part of these were withdrawn from it on the following grounds.

(1) A considerable part of the experiments, especially for freons and cryogens, was conducted in horizontal channels in which there is a possibility for the existence of regimes with periodically or continuously unwetted upper generatrices of the tube (plug, slug, stratified and wavy regimes). This may result in a sharp asymmetry of circumferentially local heat transfer coefficients and in the decrease of the section-averaged  $\alpha$ 's. The latter regimes are not considered in the present work. Flow patterns were identified with the aid of a map given in ref. [42] and based on an extensive experimental material for non-adiabatic two-phase flows. As one should expect, this new map predicts a marked expansion of the boundaries of stratified regimes in comparison with the maps based on adiabatic flow observations [91, 92]. It should be emphasized that in rather a broad range of regime parameters the new map is in good agreement

with Schicht's map modified in ref. [5] (see Fig. 1 in that work).

(2) The accuracy of measurements of the local  $T$  in two-phase channel flow strongly depends on the type of experimental set-up, sensors and instruments employed and lies within the range from 0.01 K for precision measurements on helium [15] to 1.5 K [28]. Bearing in mind the fact that the desired accuracy of the predictive correlation is at the level of, at least,  $\pm 35\%$ , the experimental data, for which the relative error in the measurements of the local  $\Delta T$ , and, hence, also of  $\alpha$ , exceed 30% were discarded as unreliable.

(3) A characteristic feature of nucleate boiling is a strong dependence of the heat transfer coefficient on the heat flux density described by the power dependence  $\alpha \sim q^n$ , where  $n$  lies in the range 0.4–0.7, in accordance with the data for channels. A diminution in this dependence for high heat fluxes recorded, for instance, in refs. [30, 43] seems to be attributed to the heat transfer crisis developing in downstream channel sections. The development or even the approach of the crisis radically changes the hydrodynamic and thermal situation in the channel, resulting, in particular, in the appearance of a large axial heat flow, which eventually leads to the apparent weakening of the  $\alpha(q)$  dependence. For this reason, the points for which  $n < 0.4$  in the region of high fluxes were ignored.

(4) As is known, in the region of low heat fluxes

or high  $G$ 's and  $x$ 's a transition to forced convection vaporization occurs, which is accompanied by the weakening of the function  $\alpha(q)$  until complete disappearance of this dependence.

Vaporization is a specific type of heat transfer in which the relationship between the heat transfer coefficient and regime parameters essentially differs from that for nucleate boiling. In order to separate the nucleate boiling region from that of two-phase convection (vaporization), use was made of the convective boiling number

$$N_{CB} = (rG/q)[1 + x(\rho_l/\rho_v - 1)](\rho_v/\rho_l)^{1/3}$$

obtained in ref. [5]. The condition  $N_{CB} < 1.6 \times 10^4$  identifies the nucleate boiling region, while the condition  $N_{CB} > 1.6 \times 10^4$  identifies the vaporization region.

Taking into consideration the restrictions considered, the total number of 2500 experimental points were processed for the region of nucleate boiling. All the experimental data used were directly obtained from the experiments for which an exhaustive primary information was available and therefore no prior data smoothing was made. Thus, the correlation given below indirectly takes into account rather a large scatter (up to  $\pm 15\%$ ) characteristic for the two-phase flow heat transfer data.

**DEVELOPMENT OF CORRELATION FOR NUCLEATE BOILING WITH FORCED CONVECTION**

The correlation is constructed in the system of dimensionless numbers which was originally suggested in ref. [2], i.e. in the form:  $Nu_b = Nu_b(Pe_*, K_p, Pr, K_\lambda)$ , with the solution being sought in a conventional form of the product of power functions

$$Nu_b = \text{const. } Pe_*^{n_1} K_p^{n_2} Pr^{n_3} K_\lambda^{n_4} \quad (4)$$

In the first place, attention was focused on finding the dependence of the heat transfer coefficient on the thermal properties of the channel wall material given by the number  $K_\lambda$ . The question about the effect of the wall material remains unsolved up to now; more-

over, the correlations suggested in refs. [1–5] are the only ones that predict the existence of such an influence. It is natural that the discovery of this effect is more probable, the broader the range of  $K_\lambda$  that varied in the experiment. Of the four groups of liquids listed in Table 1, cryogenic liquids are most preferable for research, because  $K_\lambda$  varied in a very broad range—from 14 to  $8.2 \times 10^5$ —and the tubes used in experiments were made of five different materials: silver, copper, inconel, monel and stainless steel. The treatment of experimental data from Table 1 is shown in Fig. 1, from which it follows that the effect of the thermal properties of the wall can be described by a simple power function to which there corresponds an equation of the form

$$Nu_b = 9.7 \times 10^{-3} Pe_*^{0.6} K_p^{0.5} Pr^{0.33} K_\lambda^{0.12} \quad (5)$$

Expression (5) differs from equation (2) by predicting a somewhat weaker dependence of the heat transfer coefficient on the wall material. It is necessary to emphasize that expression (5) takes into account additional experimental information accumulated over the last few years [21, 22, 24, 35, 39–42] and, besides, rests on the constantly updated bank of the thermal properties of cryogenic fluids. References [93–95] are an example of recent contributions to this bank. It should also be noted that the existence of the effect of the wall material thermal properties was experimentally demonstrated in ref. [96]. Measurements carried out in the latter work displayed an increase in the heat transfer coefficient for nitrogen nucleate boiling in a copper tube by approximately 50% compared with a stainless steel tube in the same set-up and at the same values of all regime parameters (heat flux, pressure, mass flow rate and quality). This effect corresponds exactly to the value of  $n_4 = 0.12$ .

Having thus established the degree of the influence of the wall thermal conductivity on heat transfer rate, one can turn to further improvement of the accuracy of equation (4) representing the functional relationship of the heat transfer coefficient with heat flux, pressure, and the thermal properties of the fluid and of the wall. As earlier [1–5] it will be assumed that  $n_1 = 0.6$ . This assumption reflects the fact that in channel nucleate boiling there is a weaker function

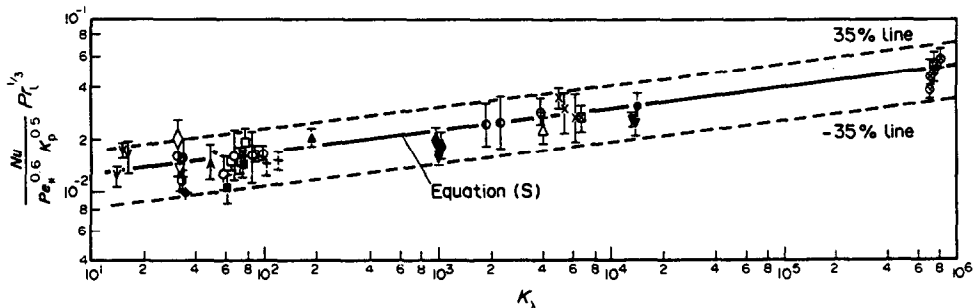


Fig. 1. Nucleate boiling heat transfer rate vs the thermal conductivity of tube wall material for cryogens in a forced flow. For symbols, see Table 1.

Table 1. Experimental investigations analysed to develop and test the proposed correlation

Ordinal No.	Authors	Fluid	Test section: dimensions (mm); orientation	Material	Pressure (bar)	Mass flow rate ( $\text{kg m}^{-2} \text{s}^{-1}$ )	Heat flux density ( $\text{kW m}^{-2}$ )	Number of experimental points		
								Vaporization $N_{\text{CB}} > 1.6 \times 10^4$	Nucleate boiling $N_{\text{CB}} < 1.6 \times 10^4$	Symbols
1	2	3	4	5	6	7	8	9	10	11
<b>Cryogenic fluids</b>										
1	La Harpe <i>et al.</i> [13]	Helium	Coiled tube: $l = 4885, d = 3.0$ ; vertical	Monel	1.2	25-120	0.021-0.56	0	13	*
2	Johannes [14]	Helium	Tube: $l = 296, d = 2.12$ ; vertical	Monel	1.11	62-230	0.145-3.2	0	18	^
3	Hildebrandt [15]	Helium	Tube: $l = 20, d = 1.0$ ; vertical	Silver	0.2-0.9	5.6-76.3	0.018-1.77	0	29	⊗
4	Ogata and Sato [16]	Helium	Tube: $l = 85, d = 1.09$ ; vertical	Stainless steel	1.1-1.9	79-163	0.040-4.0	0	35	●
5	Keilin <i>et al.</i> [17]	Helium	Tube: $l = 100, d = 2.0$ ; vertical	Copper	1.11-1.52	18-96	0.145-3.6	0	25	◆
6	Giarratano <i>et al.</i> [18]	Helium	Tube: $l = 100, d = 2.13$ ; vertical	Stainless steel	1.11	67-315	0.080-1.8	0	12	▽
7	Antipov and co-workers [19, 20]	Helium	Tube: $l = 130, d = 0.67$ ; vertical	Stainless steel	1.0	14-64	0.010-1.13	0	26	◇
8	Romanov <i>et al.</i> [21]	Helium	Tube: $l = 100, d = 0.47$ ; vertical	Copper	1.02	34	0.060-2.15	0	7	⊕
9	Anisimov [22]	Helium	Tube: $l = 130, d = 1.8$ ; vertical	Stainless steel	1.0	22-135	0.035-4.0	0	53	
10	Solodovnikov and co-workers [23, 24]	Helium	Tube: $l = 180, d = 1.63$ ; vertical	Stainless steel	1.0-1.82	65-110	0.030-1.82	8	35	V
11	Core <i>et al.</i> [25]	Hydrogen	Tube: $l = 40, d = 4.25$ ; vertical	Stainless steel	6.45	520	32.8-131	0	2	◆
12	Walters [26]	Hydrogen	Tube: $l = 152, d = 6.35$ ; horizontal	Copper	1.62-2.23	470-950	10.4-129	0	14	▼

13	Mohr and Runge [27]	Neon	Tube: $l = 100, d = 4.0$ ; horizontal	Copper	1.5	77-131	0.4-49	42	27	●	
14	Lewis <i>et al.</i> [28]	Nitrogen	Tube: $l = 144, d = 14.1$ ; vertical	Stainless steel	3.46-3.8	20.5-76.6	30.1-75.7	0	8	□	
15	Troshin and Episthov [29]	Nitrogen	Tube: $l = 900, d = 3.87$ ; horizontal	Stainless steel	0.9-1.0	92.4-199	4.74-34.5	19	10	○	
16	Klein [30]	Nitrogen	Tube: $l = 1000, d = 12$ ; horizontal	Copper	3.0	206-257	1.0-31.0	28	5	△	
17	Steiner and co-workers [31, 32]	Nitrogen	Tube: $l = 525, d = 14$ ; horizontal	Copper	5.2-20.4	117-562	0.30-37.8	18	102	×	
18	Papell and Hendricks [33]	Nitrogen	Tube: $l = 250, d = 1.98$ ; vertical	Inconel	21.7	2210	212	0	5	▲	
19	Dolgoy <i>et al.</i> [34]	Nitrogen	Tube: $l = 200, d = 1.6$ ; horizontal	Stainless steel	1.46-3.0	61-203	1.76-45.4	0	15	■	
20	Deev <i>et al.</i> [35]	Nitrogen	Tube: $l = 212, d = 2.6$ ; vertical	Stainless steel	3.4-15.0	490-2130	7.2-145	0	23	+	
21	Anisimov [22]	Nitrogen	Tube: $l = 130, d = 1.8$ ; vertical	Stainless steel	1.0	67-300	1.5-70	0	47		
22	Klimenko and Sudarchikov [36-38]	Nitrogen	Tube: $l = 1850, d = 10$ ;	Stainless steel	1.8-9.7	130-790	2.3-46	7	193	⊕	
23	Grigoriev [41]	Nitrogen	Tube: $l = 1630, d = 10$ ; vertical	Copper	1.75-4.6	242-517	3.2-46	57	104	⊙	
24	Klimenko <i>et al.</i> [39, 40]	Nitrogen	Tubes: $l = 500, d = 4, 9, 15.8$ and 18.9; vertical	Stainless steel	1.5-8.0	60-800	0.65-65	43	102	⊞	
25	Fyodorov [42]	Nitrogen	Tube: $l = 600, d = 14.1$ ; horizontal	Stainless steel	2.0-9.0	160-700	2.8-52	11	102		
26	Müller-Steinhagen <i>et al.</i> [43, 44]	Argon	Tube: $l = 175, d = 14$ ; horizontal	Copper	1.67-19.6	120-447	0.22-16	4	11	⊠	
<b>Total cryogenic fluids</b>									237	1023	

Table 1 (continued)

Ordinal No.	Authors	Fluid	Test section: dimensions (mm); orientation	Material	Pressure (bar)	Mass flow rate ( $\text{kg m}^{-2} \text{s}^{-1}$ )	Heat flux density ( $\text{kW m}^{-2}$ )	Number of experimental points		
								Vaporization $N_{\text{CH}} > 1.6 \times 10^4$	Nucleate boiling $N_{\text{CH}} < 1.6 \times 10^4$	Symbols
1	2	3	4	5	6	7	8	9	10	11
27	Bryan and Quaint [45]	Freon 11	Tube: $l = 3048, d = 8$ ; horizontal	<b>Refrigerants</b> Copper	1.2–1.6	137–587	3.17–18.1	17	0	
28	Chawla [46]	Freon 11	Tube: $l = 119, d = 6$ ; horizontal	Copper	0.61	78.7–251	4.65–93	43	24	
29	Kihara <i>et al.</i> [12]	Freon 11	Annular channel: $l = 686, d_c = 27.6$ ; vertical	Inconel	1.37	335–411	4.32–49.6	0	8	
30	Bogdanov [48]	Freon 12	Tube: $l = 1500, d = 12$ ; horizontal	Copper	4.23	273–663	3.49–23.3	0	16	
31	Uchida and Yamaguchi [49]	Freon 12	Tube: $l = 1000, d = 6.4$ ; horizontal	Stainless steel	3.92	259–518	2.44–27.9	65	7	
32	Bandel and Schlünder [51]	Freon 12	Tube: $l$ not specified, $d = 14$ ; horizontal	Copper	3.13	239–441	5.0–71	17	10	
33	Chaddock and Noerger [50]	Freon 12	Tube: $l = 1930, d = 11.7$ ; horizontal	Stainless steel	4.4	350–587	6.84–35.0	15	13	
34	Vaitinger [52]	Freon 12	Tube: $l = 1600, d = 20.5$ ; vertical	Stainless steel	11.8–33.4	860–5600	9.0–87	0	21	
35	Mayinger and Ahrens [53]	Freon 12	Tube: $l = 5000, d = 14$ ; vertical	Stainless steel	9.0–22	300–1200	10–67	0	29	
36	Malyshev <i>et al.</i> [10]	Freon 12	Tube: $l = 90, d = 10$ ; horizontal	Stainless steel	1.63	224	2.0–25	14	3	
37	Kirin <i>et al.</i> [54]	Freon 12	Tube: $l = 400, d = 8$ ; vertical	Copper	5.67	50–240	2.0	11	2	
38	Pujol and Stenning [55]	Freon 113	Tube: $l = 1295, d = 15.8$ ; vertical	Stainless steel	2.0	586	16.9–56	16	16	

39	Lazarek and Black [56]	Freon 113	Tube: $l = 126, d = 3.15$ ; vertical	Stainless steel	1.54	502	64-178	0	14	
40	Gouse and Coumou [57]	Freon 113	Tube: $l = 3810; d = 10.9$ ; horizontal	Glass	1-1.5	519-700	12.9-22.1	3	7	
41	Fagerholm <i>et al.</i> [58, 59]	Freon 114	Tubes: $l = 9000$ and $2800$ , $d = 8$ ; vertical and horizontal	Stainless steel	3.1-5.3	625-2120	15.4-84.1	5	27	
42	Altman <i>et al.</i> [60]	Freon 22	Tube: $l = 1219, d = 8.7$ ; horizontal	Copper	5.7-10.5	146-573	9.41-35.9	3	3	
43	Bogdanov [61]	Freon 22	Tube: $l = 1500, d = 12$ ; horizontal	Copper	6.8	376-626	5.8-23	0	10	
44	Kirin <i>et al.</i> [62]	Freon 22	Tube: $l = 100, d = 10$ ; vertical	Stainless steel	2.45-9.1	50-250	5-20	34	40	
45	Bartau [63]	Ammonia	Tube: $l = 474, d = 30.3$ ; vertical	Nickel	34.3-108	1000	52-1240	0	18	
<b>Total refrigerants</b>										
									243	268
<b>Organic fluids</b>										
46	Blinov <i>et al.</i> [64]	Propane	Tube: $l = 990, d = 8$ ; vertical	Stainless steel	3.0-11.5	300	4.9-135	0	16	
47	Blinov <i>et al.</i> [65]	Butane	Tube: $l = 990, d = 7.75$ ; vertical	Stainless steel	1.45	118-708	10.1-79.5	0	24	
48	Guerrieri and Talty [66]	Pentane	Tube: $l = 1981, d = 25.4$ ; vertical	Brass	1.0	165-409	17.7-38.2	0	4	
49	Guerrieri and Talty [66]	Heptane	Tube: $l = 1981, d = 25.4$ ; vertical	Brass	1.0	184-448	18.5-30	0	4	
50	Yusuphova <i>et al.</i> [67]	Isooctane	Tube: $l = 700, d = 7.2$ ; vertical	Stainless steel	2.5-20.5	684	55-490	0	23	
51	Guerrieri and Talty [66]	Benzene	Tube: $l = 1981, d = 25.4$ ; vertical	Brass	1.0	247-602	27.1-42	0	4	
52	Guerrieri and Talty [66]	Cyclohexane	Tube: $l = 1981, d = 25.4$ ; vertical	Brass	1.0	392-484	7.89-30.9	0	21	

Table 1 (continued)

Ordinal No.	Authors	Fluid	Test section: dimensions (mm); orientation	Material	Pressure (bar)	Mass flow rate ( $\text{kg m}^{-2} \text{s}^{-1}$ )	Heat flux density ( $\text{kW m}^{-2}$ )	Number of experimental points		
								Vaporization $N_{\text{CB}} > 1.6 \times 10^4$	Nucleate boiling $N_{\text{CB}} < 1.6 \times 10^4$	Symbols
1	2	3	4	5	6	7	8	9	10	11
53	Guerrieri and Talty [66]	Methanol	Tube: $l = 1981, d = 25.4$ ; vertical	Brass	1.0	225-571	18.7-54.9	0	4	
54	Lukomsky and Madorskaya [47]	Ethanol	Tube: $l = 210, d = 30$ ; vertical	Chromo-molybdenum steel	1.57-30.2	112-474	31.6-256	0	15	
<b>Total organic fluids</b>										
55	McAdams <i>et al.</i> [68]	Water	Annular channel: $l = 95.3, d_c = 47.8$ ; vertical	Stainless steel	2.07-6.21	1100-3440	237-726	0	8	
56	Sterman [69]	Water	Annular channel: $l = 90, d_c = 3.8$ ; vertical	Stainless steel	6.86	4520-6020	814	0	64	
57	Dengler and Addoms [70]	Water	Tube: $l = 6096, d = 25.4$ ; vertical	Copper	2.2	724	99.9-463	5	0	
58	Aladiyev <i>et al.</i> [71]	Water	Tubes: $l = 62.5$ and $135, d = 8.2$ ; vertical	Stainless steel	5.88-138	312-6240	523-4650	0	56	
59	Tarasova <i>et al.</i> [72, 73]	Water	Tube: $l = 350, d = 8$	Nickel	128-196	1250-3640	502-1230	0	57	
60	Averin and Kruzhiin [74]	Water	Annular channel: $l = 376, d_c = 4.61$ and $18.3$ ; vertical	Stainless steel	0.98-8.83	792-5270	815-3610	0	396	
61	Isbin <i>et al.</i> [75]	Water	Annular channel: $l = 60, d_c = 74.7$ ; vertical	Brass	1.5-5.7	248	80.8-142	26	0	
62	Bennet <i>et al.</i> [76]	Water	Tube: $l = 235, d = 9.55$ ; horizontal	Stainless steel	1.5-1.8	152-290	300-400	16	1	
63	Wright [77]	Water	Annular channel: $l = 737, d_c = 447$ and $6.12$ ; vertical	Stainless steel	1.41-4.6	667-3054	117-278	52	24	
			Tubes: $l = 1430$ and $1728, d = 12$ and $18.3$ ; vertical							



64	Alexeev <i>et al.</i> [78]	Water	Tube: $l = 700, d = 9$ ; vertical	Nickel	29.4–147	250–2000	98.8–692	0	54
65	Morozov [79]	Water	Tube: $l = 250, d = 32$ ; vertical	Stainless steel	30.4–40	116–515	112–515	0	100
66	Collier <i>et al.</i> [80]	Water	Annular channel: $l = 1219, d_c = 6.12$ ; vertical	Stainless steel	1.6	136–258	199–401	10	7
67	Rounthwaite [81]	Water	Tube: $l = 6248; d = 41.3$ ; horizontal	Mild steel	14.8	235	63.1	14	0
68	Bertoletti <i>et al.</i> [8]	Water	Tube: $l = 1399, d = 9.8$ ; vertical	Stainless steel	70	1100–3800	200–1980	0	39
69	Thom <i>et al.</i> [82]	Water	Tube: $l = 127, d = 12.7$ Annular channel: $l = 305, d_c = 11.6$ ; vertical	Stainless steel	52.8–138	1040–3800	284–1670	0	23
70	Müller [83]	Water	Tube: $l = 240, d = 13.1$ ; vertical	Stainless steel	1.52–181	609–5700	175–8000	0	168
71	Hodgson [84]	Water	Tube: $l = 152, d = 11.7$ ; vertical	Stainless steel	13.8–34.5	1780	1913	0	21
72	Andreyevsky <i>et al.</i> [85, 86]	Water	Tubes: $l = 800, d = 8.12$ and 18; vertical	Stainless steel	4.1–30.4	183–2690	352–1490	75	58
73	Herkenrath and Mörk- Mörkenstein [87]	Water	Tube: $l = 5100, d = 10$ ; vertical	Stainless steel	205	1500	600	0	3
74	Cheng and co-workers [9, 88]	Water	Tubes: $l = 50.8$ and 108, $d = 12.7$ ; vertical	Copper Inconel	1.01	136	470–1400	0	8
75	Kenning and co-workers [89, 90]	Water	Tube: $l = 500, d = 9.6$ ; vertical	Copper- nickel alloy	1.0–3.9	105–304	50–400	37	7
Total water								235	1094
Grand total								715	2500
Total number of experimental points								3215	

$\alpha(q)$  than in pool boiling. An assumption that the expression  $\alpha \sim \mu_1^{-0.33}$  [97] describes the influence of viscosity on the heat transfer coefficient yields  $n_3 = -0.33$ . The exponent at dimensionless pressure  $K_p$  was found on the basis of all the data given in Table 1. It appeared to be the same for all the groups of fluids and equal to  $n_2 = 0.54$ . Hence, the final correlation for calculating the heat transfer for the forced flow nucleate boiling takes the form

$$Nu_b = C Pe_*^{0.6} K_p^{0.54} Pr_1^{-0.33} K_\lambda^{0.12}. \quad (6)$$

The constant  $C$  in equation (6) turns out to be different for different groups of fluids and can be found from Table 2 given below.

The fact that dimensionless equation (6) for the heat transfer coefficient contains an individual constant for each group of fluids may seem at first glance illogical. However, it is explained by a number of objective reasons, the most important of which are the following.

(1) The system of dimensionless parameters (4) is not complete because it lacks variables characterizing the channel inner wall roughness ( $R_a/b$ ) [2], wettability ( $\theta$ ), individual properties of substance ( $A$  or some other parameter of thermodynamic similarity).

(2) The influence of the above listed parameters is rather weak compared with the influence of, say,  $q$  or  $p$ , and which is of no less importance, is rather uncertain. Moreover, there has always been and will continue to exist the shortage of information about the quantities  $R_a$  and  $\theta$ .

(3) Nobody has ever experimentally investigated the dependence of the heat transfer coefficient on viscosity in nucleate boiling under the conditions of forced flow. That is why the exponent of a power  $n_3 = -0.33$  is likely to be the least substantiated one of all those entering into expression (6). It is of interest that a certain correlation between the value of the constant in equation (6) and Prandtl numbers for separate groups of fluids (see Table 2) is observed. However, it is hardly probable that at the moment there is sufficient information for drawing a responsible conclusion.

Taking into account the above, the appearance of an individual constant in equation (6) seems to be quite natural now.

Finally, it should be noted that the attempts to correlate the data on pool boiling, that made claims to a sufficiently high accuracy, have always led to the

appearance of individual constants or even equations not only for separate groups of fluids, but also for individual combinations of liquid-heating surface [98–100]. Equation (6) predicts a markedly weaker dependence of heat transfer coefficient on heat flux and pressure in nucleate boiling under the conditions of forced flow compared with boiling in a pool. It is most clearly seen if written side by side with one of the most well-known equations for boiling in a pool, i.e. Kutateladze's equation [101]

$$Nu_b = 1.5 \times 10^{-3} Pe_*^{2/3} K_p^{2/3} (1 - \rho_v/\rho_l)^{2/3}. \quad (7)$$

The reasons for such a discrepancy were discussed in detail earlier [4].

Completing the consideration of the correlation obtained, it is worth adding that, to simplify the calculations of nucleate boiling heat transfer rate for water, one can use the following equation:

$$\alpha = \frac{9.6p^{0.14}}{1 - 4 \times 10^{-3}p} q^{0.6} \quad (8)$$

where  $\alpha$ ,  $q$  and  $p$  are measured in  $W m^{-2} K^{-1}$ ,  $W m^{-2}$  and bar, respectively. The coefficient 9.6 corresponds to boiling in a channel with metallic walls, with their thermal conductivity being  $\lambda_w = 14\text{--}20 W m^{-1} K^{-1}$  (stainless steel, inconel, titanium and zirconium alloys). Equation (8) approximates correlation (6) in the range of  $p = 1\text{--}200$  bar within 11%. According to equation (6), for water boiling in a nickel tube, the coefficient in equation (8) increases up to 11.2 and in a copper unoxidized tube increases up to 13.6.

In Fig. 2 calculations by equation (6) for water are compared with calculations by Rassokhin *et al.*'s equation [102]

$$\alpha = \begin{cases} 3.29q^{2/3}p^{1/4} & \text{for } 1 < p < 80 \text{ bar} \\ 0.0288q^{2/3}p^{1/3} & \text{for } 80 < p < 200 \text{ bar} \end{cases} \quad (9)$$

which was recommended [103] as the best equation for representing the experimental data. As is seen from Fig. 2, a good agreement of calculations made by equation (6) and by equations (9) is observed: the deviation never exceeds 40% and in the majority of cases it lies within 20%. This fact is evidence of the correctness of the analysis undertaken above.

#### CALCULATION RECOMMENDATIONS AND COMPARISON WITH EXPERIMENTAL DATA

To calculate heat transfer to a two-phase flow in vertical and horizontal channels in the case when the perimeter is fully wetted by a liquid, the correlation given below can be used:

$$Nu_{TP} = \max \{ Nu_b; Nu_c \} \quad (10)$$

where  $Nu_b$  and  $Nu_c$  are found from equations (6) and (3), respectively.

Table 2

Fluids	$C$	$Pr$
Freons	$7.6 \times 10^{-3}$	$3.8 \pm 2.5$
Organic fluids	$6.8 \times 10^{-3}$	$3.5 \pm 1.3$
Cryogenic fluids	$6.1 \times 10^{-3}$	$1.6 \pm 1.0$
Water	$4.9 \times 10^{-3}$	$1.2 \pm 0.4$

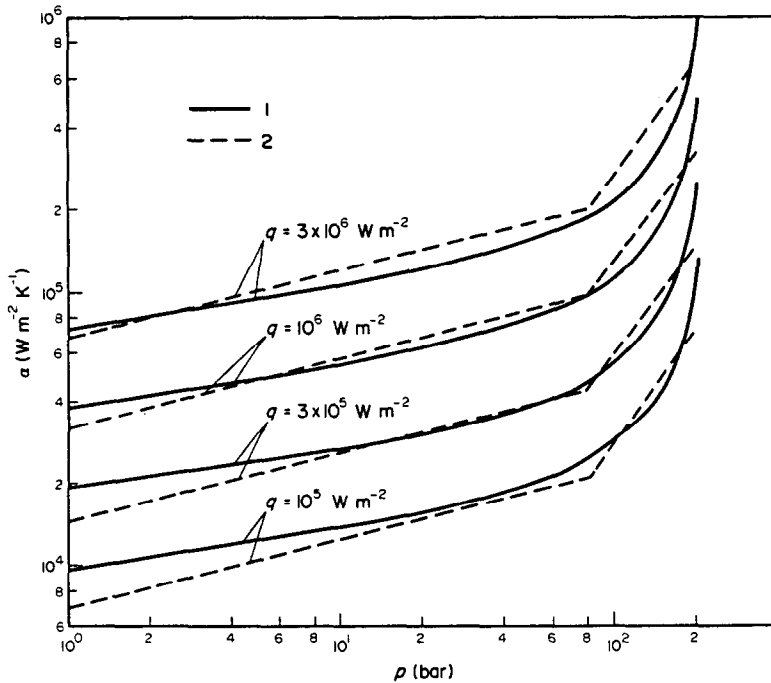


Fig. 2. Comparison of the water nucleate boiling heat transfer rate under conditions of forced convection calculated by different correlations: 1, equation (6) (tube material, stainless steel); 2, equation (9).

The position of the boundary between the regions of the predominant influence of nucleate boiling and two-phase forced convection is defined by the condition  $N_{CB} = 1.6 \times 10^4$  with an accuracy within  $\pm 25\%$ . The latter means that in the case when  $N_{CB}$  numbers are beyond the range of  $1.2 \times 10^4 - 2.0 \times 10^4$ , the need for choosing the maximum heat transfer coefficient out of the two coefficients is avoided, and the calculation can be made directly either by expression (6) (with  $N_{CB} < 1.2 \times 10^4$ ), or by expression (3) (with  $N_{CB} > 2.0 \times 10^4$ ). For water nucleate boiling equation (6) can be replaced by dimensional formula (8) which approximates it. Equations (3) and (6) show that in two-phase flow the heat transfer is independent of the channel diam-

eter. This statement is valid if the condition  $(d/b) > 1.5$  is satisfied [1].

Tables 3–5 list the results of comparison of all the experimental data from Table 1 with correlation (10). Altogether 3215 experimental points on two-phase flow heat transfer were analysed (2500 for the nucleate boiling region and 715 for the vaporization region). Out of this number, only 96 points, i.e. 3%, deviate from predictive correlation (10) by more than 35% (82 points for nucleate boiling and only 14 points for vaporization). The average absolute deviation given by the relation

$$\bar{D} = \frac{1}{N} \sum_{i=1}^N \left( \frac{|\alpha_{exp} - \alpha_{calc}|}{\alpha_{calc}} \right) \quad (11)$$

Table 3. Comparison of experimental data with prediction. Nucleate boiling

Fluid	Number of fluids investigated	Total number of experimental points	Parameter			
			Number of points outside the $\pm 35\%$ range (percentage of the total number)	Number of points outside the 73% to $-42\%$ range (percentage of the total number)	Mean absolute deviation, $\bar{D}$ (%)	Maximum deviation $\frac{\alpha_{exp} - \alpha_{calc}}{\alpha_{calc}}$ (%)
Cryogenes	5	1023	2.1	0.1	13.6	68 and $-43$
Freons	6	268	6.0	1.1	19.3	51 and $-56$
Organic fluids	9	115	1.7	0.0	13.5	48 and $-35$
Water	1	1094	3.9	0.7	14.3	64 and $-65$
All fluids	21	2500	3.3	0.5	14.5	68 and $-65$

Table 4. Comparison of experimental data with prediction. Forced convection vaporization

Fluid	Parameter					
	Number of fluids investigated	Total number of experimental points	Number of points outside the $\pm 35\%$ range (percentage of the total number)	Number of points outside the 73% to $-42\%$ range (percentage of the total number)	Mean absolute deviation, $\bar{D}$ (%)	Maximum deviation $\frac{\alpha_{\text{exp}} - \alpha_{\text{calc}}}{\alpha_{\text{calc}}}$ (%)
Cryogenes	4	237	0.8	0.0	13.8	39 and $-35$
Freons	5	243	2.9	0.0	14.5	59 and $-34$
Organic fluids	0	0	0.0	0.0	0.0	0
Water	1	235	2.1	0.0	13.3	46 and $-25$
All fluids	10	715	2.0	0.0	13.9	59 and $-35$

Table 5. Comparison of experimental data with prediction. Two-phase flow heat transfer

Fluid	Parameter					
	Number of fluids investigated	Total number of experimental points	Number of points outside the $\pm 35\%$ range (percentage of the total number)	Number of points outside the 73% to $-42\%$ range (percentage of the total number)	Mean absolute deviation, $\bar{D}$ (%)	Maximum deviation $\frac{\alpha_{\text{exp}} - \alpha_{\text{calc}}}{\alpha_{\text{calc}}}$ (%)
Cryogenes	5	1260	1.8	0.1	13.6	68 and $-43$
Freons	6	511	4.1	0.6	17.0	59 and $-56$
Organic fluids	9	115	1.7	0.0	13.5	48 and $-35$
Water	1	1329	3.6	0.6	14.1	64 and $-65$
All fluids	21	3215	3.0	0.4	14.4	68 and $-65$

in which  $N$  is the number of experimental points, amounted to 14.4% which is much better than for any other correlations (see for instance, Table 2 from ref. [5]).

Tables 3–5 include, besides three rather often used deviation parameters, one more parameter, i.e. a number of points beyond the interval 73% to  $-42\%$ . Its introduction is based on the following arguments. It is postulated that in the calculation of such a complex process as the two-phase flow heat transfer, the scale of 'quality' of the predictive correlation might look as follows:

$\bar{D} < 15\%$ —a very good one (Chen's relationship, for instance, agrees with the data from refs. [66, 70, 76, 104, 105], on the basis of which it was derived);

$\bar{D} = 15\text{--}20\%$ —a good one (Bjorge *et al.*'s relationship [106] compared with the data on water from eight references);

$\bar{D} = 20\text{--}25\%$ —acceptable (Chen's relationship compared with available data on forced convection vaporization, see ref. [5]);

$\bar{D} > 25\%$ —unacceptable (any correlation, known to the present author, compared with the data from Table 1).

The mean deviation of 25% implies that approximately 95% of the points from the described bulk of the experimental data lie within the range of  $\pm 50\%$  of the calculated one, i.e. the ratio of the maximum and minimum heat transfer coefficients lying in this interval constitutes  $\alpha_{\text{max}}/\alpha_{\text{min}} = 1.5/0.5 = 3.0$ . When dealing with rather large deviations it would be more correct, in the author's opinion, to operate not with percentage of deviation, but rather with the value showing by how many times the experiment deviates from the calculation. Otherwise, the deviation equal, say, to 80% can be considered to be 'worse' than that equal to  $-50\%$ , in spite of the fact that in the second case the disagreement is two-fold, while in the first case it is only 1.8-fold. Thus, to large deviations there should correspond an 'asymmetric' error band. If one assumes that  $\alpha_{\text{max}}/\alpha_{\text{min}} = 3.0$  ( $\bar{D} \approx 25\%$ ), then in the range 73% to  $-42\%$  one will find data deviating from the calculation by not more than  $\sqrt{3}$  times. Let us assume that the data beyond this range are not represented by that correlation. For the correlation considered the number of such data is very small. They include:

(a) data on water boiling at  $p = 205$  bar [87], as well as some points from refs. [68, 88];

(b) data on ammonia boiling at  $p = 108$  bar [63].

It is not difficult to conclude that basically the proposed correlation overpredicts the data which are obtained at high reduced pressure ( $p/p_{cr} > 0.92$ ). It should be noted that the error of the experiments under such pressures is rather high ( $\Delta T < 1$  K) and the information about the thermal properties of the fluid is rather inaccurate. Additional and precise experiments are required to specify with certainty the character of the function  $\alpha(p)$  in the vicinity of the critical point. At the Moscow Power Engineering Institute a special programme is being carried out aimed at the research of forced flow nucleate boiling under high reduced pressures.

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#### REFERENCES

1. V. V. Klimenko, Heat transfer intensity at forced flow boiling of cryogenic liquids in tubes, *Cryogenics* **22**(1), 569–576 (1982).
2. V. V. Klimenko and A. V. Grigoriev, An approximate theory of nucleate flow boiling in tubes, *Izv. Akad. Nauk SSSR, Energetika Transport* No. 6, 116–125 (1983).
3. V. V. Klimenko, Heat transfer in forced convection boiling in a channel, *Proc. 7th All-Union Heat and Mass Transfer Conf.*, Minsk, Vol. 4, pt. 1, pp. 99–103 (1984); *Heat Transfer—Sov. Res.* **17**(2), 96–100 (1985).
4. V. A. Grigoriev, V. V. Klimenko and A. V. Klimenko, Pool and channel nucleate boiling of cryogenic fluids—similarities and differences, *Proc. 8th Int. Heat Transfer Conf.*, San Francisco, Vol. 5, pp. 2161–2166 (1986).
5. V. V. Klimenko, A generalized correlation for two-phase forced flow heat transfer, *Int. J. Heat Mass Transfer* **31**, 541–552 (1988).
6. J. C. Chen, Correlation for boiling heat transfer to saturated liquids in convective flow, *Ind. Engng Chem. Process Des. Dev.* **5**(3), 322–329 (1966).
7. M. M. Shah, A new correlation for heat transfer during boiling flow through pipes, *ASHRAE Trans.* **82**(2), 66–86 (1976).
8. S. Bertoletti, C. Lombardi and M. Silvestri, Heat transfer to steam–water mixture, CISE Rept. R-78 (1964).
9. W. W. L. Ng and S. C. Cheng, Steady state flow boiling curve measurements via temperature controller, *Lett. Heat Mass Transfer* **6**(1), 77–81 (1979).
10. A. A. Malyshev, G. N. Danilova, V. M. Azarskov and B. B. Zemskov, Influence of flow patterns of two-phase R 12 flow on boiling heat transfer in horizontal tubes, *Kholodilnaya Tekhnika* No. 8, 30–34 (1982).
11. J. Bandel and E. U. Schlünder, Frictional pressure drop and convective heat transfer of gas–liquid flow in horizontal tubes, *Proc. 5th Int. Heat Transfer Conf.*, Tokyo, Vol. 5, pp. 190–194 (1974).
12. D. H. Kihara, H. C. Chai and A. Ching, Jr., Forced convection boiling heat transfer of Freon-11 in a vertical annular passage, *Lett. Heat Mass Transfer* **6**(1), 13–21 (1979).
13. A. de La Harpe, S. Lehongre, J. Mollard and C. Johannes, Boiling heat transfer and pressure drop of liquid helium-I under forced circulation in a helically coiled tube, *Adv. Cryogen. Engng* **14**, 170–177 (1969).
14. C. Johannes, Studies of forced convection heat transfer to helium-I, *Adv. Cryogen. Engng* **17**, 352–360 (1972).
15. C. Hildebrandt, Heat transfer to boiling helium-I under forced flow in a vertical tube, *Proc. 5th Int. Cryogen. Engng Conf.*, Kyoto, pp. 295–300 (1974).
16. H. Ogata and S. Sato, Forced convection heat transfer to boiling helium in a tube, *Cryogenics* **14**(7), 375–380 (1974).
17. V. E. Keilin, I. A. Kovalev, V. V. Likov and M. M. Pozvonkov, Forced convection heat transfer to liquid helium-I in the nucleate boiling region, *Cryogenics* **15**(3), 141–145 (1975).
18. P. J. Giarratano, R. C. Hess and M. C. Jones, Forced convection heat transfer to subcritical helium-I, *Adv. Cryogen. Engng* **19**, 404–416 (1974).
19. V. A. Grigoriev, V. I. Antipov, Yu. M. Pavlov and A. V. Klimenko, An experimental investigation of boiling nitrogen and helium heat transfer in channels, *Teploenergetika* No. 4, 11–14 (1977).
20. V. I. Antipov, Investigation of heat transfer of boiling cryogenic fluids in small diameter tubes under the conditions of forced flow, Ph.D. Thesis, Moscow Power Engineering Institute (1978).
21. V. I. Romanov, A. L. Sevryugin, Yu. M. Pavlov and V. I. Antipov, Investigating burnouts with helium boiling in a channel, *Thermal Engng* **28**(10), 620–622 (1981).
22. S. B. Anisimov, Forced flow boiling heat transfer of cryogenic fluids in a rotating channel, Ph.D. Thesis, Institute for High Temperatures, Moscow (1985).
23. R. N. Yusupov, V. I. Deev, V. V. Arkhipov and V. V. Solodovnikov, Investigation of heat transfer in two-phase flow of helium. In *Numerical and Experimental Methods in Thermal Physics of Nuclear Reactors*, pp. 67–72. Moscow Engineering Physics Institute, Moscow (1982).
24. V. V. Solodovnikov, Thermohydraulic characteristics of two-phase flow of helium in vertical tubes, Ph.D. Thesis, Moscow Engineering Physics Institute, Moscow (1986).
25. T. C. Core, J. F. Harkee, B. Misra and K. Sato, Heat transfer studies, WADD-60-239, Aerojet-General Corp. (1959).
26. H. H. Walters, Single-tube heat transfer tests with liquid hydrogen, *Adv. Cryogen. Engng* **6**, 509–516 (1961).
27. V. Mohr and R. Runge, Forced convection boiling of neon in horizontal tubes. In *Heat Transfer in Boiling* (Edited by E. Hahne and U. Grigull), pp. 307–343. Hemisphere, Washington, DC (1977).
28. J. P. Lewis, J. H. Goodykoontz and J. F. Kline, Boiling heat transfer to liquid hydrogen and nitrogen in forced flow, NASA TN D-1314 (1962).
29. A. K. Troshin and V. Z. Episthov, Two-phase forced flow heat transfer and hydraulic resistance in tubes, *Kriogennoe i Kislородnoe Mashinostroeniye* No. 3, 19–22 (1974).
30. G. Klein, Heat transfer for evaporating nitrogen streaming in a horizontal tube, *Proc. 6th Int. Cryogen. Engng Conf.*, Grenoble, pp. 314–318 (1976).
31. D. Steiner and E. U. Schlünder, Heat transfer and pressure drop for boiling nitrogen flowing in a horizontal tube. In *Heat Transfer in Boiling* (Edited by E. Hahne and U. Grigull), pp. 263–306. Hemisphere, Washington, DC (1977).
32. H. M. Müller, W. Bonn and D. Steiner, Flow boiling heat transfer and pressure drop of nitrogen from 1 to 30 bar, *Proc. 9th Int. Cryogen. Engng Conf.*, Kobe, pp. 69–72 (1982).

33. S. S. Papell and R. C. Hendricks, Boiling incipience and convective boiling of neon and nitrogen, *Adv. Cryogen. Engng* **23**, 284–294 (1978).
34. M. L. Dolgoy, A. M. Troyanov and Yu. A. Puzirkov, Investigation of nitrogen forced flow heat transfer in a horizontal channel. In *Heat Transfer at Low Temperatures*, pp. 25–32. Izd. Naukova Dumka, Kiev (1979).
35. V. I. Deev, V. V. Arkhipov and V. N. Novikov, Heat transfer of nitrogen boiling under forced flow conditions. *Teploenergetika* No. 3, 26–29 (1984).
36. V. V. Klimenko and A. M. Sudarchikov, Investigation of forced flow boiling of nitrogen in a long vertical tube, *Cryogenics* **23**(7), 379–385 (1983).
37. V. V. Klimenko and A. M. Sudarchikov, Heat transfer of the forced flow of liquid nitrogen in a channel, *Trans. Moscow Pwr Engng Inst.* No. 615, 34–39 (1983).
38. V. V. Klimenko and A. M. Sudarchikov, Some features of nucleate boiling on the inner surface of a tube, *Trans. Moscow Pwr Inst.* No. 615, 40–45 (1983).
39. V. V. Klimenko, Yu. A. Fomichyov and A. V. Grigoriev, An experimental investigation of heat transfer with evaporation of liquid nitrogen in a vertical channel, *Thermal Engng* **34**(9), 493–497 (1987).
40. V. V. Klimenko, M. V. Fyodorov and Yu. A. Fomichyov, Channel orientation and geometry influence on heat transfer with two-phase forced flow of nitrogen, *Cryogenics* **29**(1), 31–36 (1989).
41. A. V. Grigoriev, Theoretical and experimental investigation and development of heat transfer predictive technique for nucleate boiling of cryogenic fluids in forced convection, Ph.D. Thesis, Moscow Power Engineering Institute, Moscow (1984).
42. M. V. Fyodorov, Heat transfer of a two-phase flow of cryogenic fluid in a horizontal channel. Experimental investigation and development of generalized predictive technique, Ph.D. Thesis, Moscow Power Engineering Institute, Moscow (1989).
43. H. M. Müller, C. Hechler, D. Steiner and E. U. Schlünder, "Fouling"—Vorgänge bei der Verdampfung von Argon in Waagrecht durchströmten Rohr, *Wärme- und Stoffübertr.* **17**(1), 47–53 (1982).
44. H. M. Müller-Steinhagen, Wärmeübergang und Fouling beim Strömungssieden von Argon und Stickstoff im Horizontalen Rohr, *Fortschz-Ber. VDI-Z* **6**(143), 1–252 (1984).
45. W. L. Bryan and G. W. Quaint, Heat transfer coefficients in horizontal tube evaporators, *Refrig. Engng* **59**(1), 67–72 (1951).
46. J. M. Chawla, Wärmeübergang und Drückabfall im waagerechten Rohren bei der Strömung von verdampfenden Kaltemitteln, *VDI ForschHft* **523**, 5–36 (1967).
47. S. M. Lukomsky and S. M. Madorskaya, Heat transfer of a boiling ethanol in tubes under the conditions of natural circulation, *Izv. Akad. Nauk SSSR, OTN* No. 9, 1306–1320 (1951).
48. S. N. Bogdanov, A study of heat transfer with Freon-12 boiling inside a horizontal tube, *Kholodilnaya Tekhnika* No. 5, 31–35 (1963).
49. H. Uchida and S. Yamaguchi, Heat transfer in two-phase flow of Refrigerant 12 through horizontal tube, *Proc. 3rd Int. Heat Transfer Conf.*, Chicago, Vol. 5, pp. 69–79 (1966).
50. J. B. Chaddock and J. A. Noerager, Evaporation of Refrigerant 12 in horizontal tube with constant heat flux, *ASHRAE Trans.* **72**(1), 99–103 (1966).
51. J. Bandel and E. U. Schlünder, Frictional pressure drop and convective heat transfer of gas-liquid flow in horizontal tubes, *Proc. 5th Int. Heat Transfer Conf.*, Tokyo, Vol. 5, pp. 190–194 (1974).
52. D. Vaihinger, The increase of nucleate boiling heat transfer with pressure. In *Heat Transfer in Boiling* (Edited by E. Hahne and U. Grigull), pp. 249–261. Hemisphere, Washington, DC (1977).
53. F. Mayinger and K. H. Ahrens, Boiling heat transfer in the transition region from bubble flow to annular flow. In *Two-phase Momentum, Heat and Mass Transfer in Chemical, Process and Energy Engineering Systems*, Vol. 2, pp. 591–602. Hemisphere, Washington, DC (1979).
54. V. K. Kirin, B. B. Zemskov and A. A. Malyshev, An effect of tube orientation and metallic coating on in-tube boiling heat transfer of R 12. In *Kriogennaya Tekhnika i Konditsionirovaniye*, pp. 99–104. Leningrad (1984).
55. L. Pujol and A. H. Stenning, Effect of flow direction on the boiling heat transfer coefficient in vertical tubes. In *Cocurrent Gas-Liquid Flow*, pp. 401–453. Plenum Press, New York (1969).
56. G. M. Lazarek and S. H. Black, Evaporative heat transfer, pressure drop and critical heat flux in a small vertical tube with R-113, *Int. J. Heat Mass Transfer* **25**, 945–960 (1982).
57. S. W. Gouse and K. G. Coumou, Heat transfer and fluid flow inside a horizontal tube evaporator, *ASHRAE Trans.* **71**(2), 152–160 (1965).
58. N.-E. Fagerholm, A. R. Ghazanfari and K. Kivioja, Boiling heat transfer in a vertical tube with Freon 114, *Wärme- und Stoffübertr.* **17**(4), 221–232 (1983).
59. N.-E. Fagerholm, A. R. Ghazanfari and K. Kivioja, Heat transfer and pressure drop of boiling R 114 in a horizontal tube, *Proc. 8th Int. Heat Transfer Conf.*, San Francisco, Vol. 5, pp. 2167–2171 (1986).
60. M. Altman, R. H. Norris and F. W. Staub, Local average heat transfer and pressure drop for refrigerants evaporating in horizontal tubes, *Trans. ASME, J. Heat Transfer* **80**(3), 189–198 (1960).
61. S. N. Bogdanov, A study of heat transfer of freons boiling inside a horizontal tube, *Coll. Pap. Central Boiler Turbine Res. Inst.* No. 57, 81–87 (1965).
62. V. K. Kirin, G. N. Danilova, V. M. Azarskov and B. B. Zemskov, Heat transfer in two-phase forced vertical flow of R 22, *Kholodilnaya Tekhnika* No. 1, 44–49 (1986).
63. G. Bartau, Experimental investigation of ammonia flow boiling at high pressures, *Proc. 5th All-Union Heat and Mass Transfer Conf.*, Minsk, Vol. 3, pt. 1, pp. 220–225 (1976).
64. V. V. Blinov, A. D. Dvoyris, L. S. Midler and O. A. Benyaminovich, Heat transfer of propane boiling in tubes under forced convection, *Gazovaya Promyshlennost* No. 10, 41–44 (1970).
65. V. V. Blinov, A. D. Dvoyris, L. S. Midler and O. A. Benyaminovich, Experimental investigation of heat transfer of boiling n-butane in vertical tubes, *J. Engng Phys.* **17**(4), 616–621 (1969).
66. S. A. Guerrieri and R. D. Talty, A study of heat transfer to organic liquids in single-tube, natural circulation vertical tube boilers, *Chem. Engng Prog. Symp. Ser.* **52**(18), 69–77 (1956).
67. V. D. Yusuphova, I. G. Danishyan, A. I. Chernyakhovsky and T. G. Ramazanov, A study of heat transfer process and critical heat loads for boiling organic fluids in tubes. In *Heat Transfer Crises and Near-critical Region* (Edited by V. M. Borishansky), pp. 58–63. Izd. Nauka, Leningrad (1977).
68. W. H. McAdams, W. E. Kennel, C. S. L. Minden, R. Carl, P. M. Picornell and J. E. Dew, Heat transfer at high rates to water with surface boiling, *Ind. Engng Chem.* **41**(9), 1945–1953 (1949).
69. L. S. Sterman, A study of heat transfer with boiling liquid in tubes, *Zh. Tekh. Fiz.* **24**(11), 2046–2053 (1954).
70. C. E. Dengler and J. N. Addoms, Heat transfer mechanism for vaporization of water in a vertical tube, *Chem. Engng Prog. Symp. Ser.* **52**(18), 95–103 (1956).

71. I. T. Aladiyev, L. D. Dodonov and V. S. Udalov, Heat transfer with boiling subcooled water in tubes, *Teploenergetika* No. 9, 64–67 (1957).
72. N. V. Tarasova, A. A. Armand and A. S. Kon'kov, An investigation of heat transfer in tube with boiling subcooled water and steam–water mixture. In *Heat Transfer at High Heat Loads and Other Special Conditions*, pp. 6–22. Energoizdat, Moscow–Leningrad (1959).
73. N. V. Tarasova and V. M. Orlov, Heat transfer and hydraulic resistance during surface boiling of water in annular channel. In *Convective Heat Transfer in Two-phase and One-phase Flows* (Edited by V. M. Borishansky and I. I. Paleev), pp. 162–187. Izd. Energiya, Moscow–Leningrad (1964).
74. E. K. Averin and G. N. Kruzhilin, Heat transfer with boiling water under conditions of forced circulation. In *Heat Transfer and Thermal Modelling*, pp. 239–270. Akademizdat, Moscow (1959).
75. H. S. Isbin, A. Kvamme, Y. Yamazaki, I. Garcia, Jr. and C. Stendahl, Heat transfer to steam–water flows, *Proc. 1961 Heat Transfer and Fluid Mechanics Inst. of Stanford Univ.*, pp. 70–78. Stanford University Press, Stanford, California (1961).
76. J. A. R. Bennett, J. G. Collier, H. R. C. Pratt and J. D. Thornton, Heat transfer to two-phase gas–liquid systems. Part I: steam–water mixtures in the liquid-dispersed region in an annulus, *Trans. Instn Chem. Engrs* **39**, 113–126 (1961).
77. R. M. Wright, Downward forced convection boiling of water in uniformly heated tubes. UCRL-9744 (1961).
78. G. V. Alexeev, B. A. Zenkevich and V. I. Subbotin, A study of nucleate boiling heat transfer of water in tubes, *Teploenergetika* No. 4, 74–77 (1962).
79. V. G. Morozov, Heat transfer during the boiling of water in tubes. In *Convective Heat Transfer in Two-phase and One-phase Flows* (Edited by V. M. Borishansky and I. I. Paleev), pp. 130–139. Izd. Energiya, Moscow–Leningrad (1964).
80. J. G. Collier, P. M. Lacey and D. J. Pulling, Heat transfer to two-phase gas–liquid systems. Part II: further data on steam/water mixtures in the liquid dispersed region in an annulus, *Trans. Instn Chem. Engrs* **42**, 127–139 (1964).
81. C. Rounthwaite, Two-phase heat transfer in horizontal tubes, *J. Inst. Fuel* **41**(325), 66–76 (1968).
82. J. R. Thom, W. M. Walker, T. A. Fallon and G. F. S. Reising, Boiling in subcooled water during flow up heated tubes or annuli, *Proc. Instn Mech. Engrs* **180**(3C), 226–246 (1965–1966).
83. F. Müller, Wärmeübergang bei der Verdampfung unter hohen Drücken, *VDI ForschHft* **522**, 5–48 (1967).
84. A. S. Hodgson, Forced convection, subcooled boiling heat transfer with water in an electrically heated tube at 100–550 lb/in<sup>2</sup>, *Trans. Instn Chem. Engrs* **46**, 25–31 (1968).
85. A. A. Andreyevsky, V. M. Borishansky, A. G. Kryuchkov, I. B. Gavrilov, G. P. Danilova, V. N. Fromzel and B. S. Fokin, Cooling of heated surface with two-phase steam–water flow, *Coll. Pap. Central Boiler Turbine Res. Inst.* No. 86, 3–25 (1968).
86. V. M. Borishansky, A. A. Andreyevsky, A. G. Kryuchkov, B. S. Fokin, V. N. Fromzel, I. B. Gavrilov and G. P. Danilova, Heat transfer to two-phase flow, *Teploenergetika* No. 5, 58–61 (1969).
87. H. Herkenrath and P. Mörk-Mörkenstein, Die Wärmeübergangskrise von Wasser bei erzwungener Strömung unter hohen Drücken. *Atomkernenergie* **14**, 163–170 (1969).
88. S. C. Cheng, W. W. L. Ng and K. T. Heng, Measurements of boiling curves of subcooled water under forced convective conditions, *Int. J. Heat Mass Transfer* **21**, 1385–1392 (1978).
89. Y. Aounallah, D. B. R. Kenning, P. B. Whalley and G. F. Hewitt, Boiling heat transfer in annular flow, *Proc. 7th Int. Heat Transfer Conf.*, Munich, Vol. 4, pp. 193–199 (1982).
90. D. B. R. Kenning and G. F. Hewitt, Boiling heat transfer in the annular flow regime, *Proc. 8th Int. Heat Transfer Conf.*, San Francisco, Vol. 5, pp. 2185–2190 (1986).
91. H. H. Schicht, Flow patterns for an adiabatic flow of water and air within a horizontal tube, *Verfahrenstechnik* **3**(4), 153–161 (1969).
92. J. M. Mandhane, G. A. Gregory and K. Aziz, A flow pattern map for gas–liquid flow in horizontal pipes, *Int. J. Multiphase Flow* **1**(4), 537–553 (1974).
93. V. V. Sytchev, A. A. Vasserman, A. D. Kozlov, G. A. Spiridonov and V. A. Tsymarny, *Thermodynamic Properties of Helium*. Izd. Standartov, Moscow (1984).
94. M. Iino, M. Suzuki and A. J. Ikushima, Surface tension of liquid <sup>4</sup>He. Surface energy of the Bose–Einstein condensate, *J. Low Temp. Phys.* **61**(1/2), 155–169 (1985).
95. N. M. Semenova and G. V. Yermakov, The boundary of stability and the surface tension of liquid helium-4, *Teplofiz. Vysok. Temp.* **24**(6), 1225–1227 (1986).
96. V. V. Klimentko, A. V. Grigoriev and A. M. Sudarchikov, The effect of heating surface material on nucleate boiling heat transfer with forced convection in channels, *Dokl. Akad. Nauk SSSR* **283**(1), 114–116 (1985).
97. F. Mayinger and E. Holborn, The effect of liquid viscosity on bubble formation and heat transfer in boiling. In *Heat Transfer in Boiling* (Edited by E. Hahne and U. Grigull), pp. 391–424. Hemisphere, Washington, DC (1977).
98. W. M. Rohsenow, A method of correlating heat transfer data for surface boiling of liquids, *Trans. ASME* **74**(8), 969–976 (1952).
99. V. M. Borishansky, G. N. Danilova, M. A. Gotovsky, A. V. Borishanskaya, G. P. Danilova and A. V. Kupriyanova, A generalization of heat transfer and elementary characteristics of the process with nucleate boiling. In *Heat Transfer and Hydrodynamics* (Edited by V. M. Borishansky), pp. 54–71. Izd. Nauka, Leningrad (1977).
100. K. Stephan and M. Abdelsalam, Heat-transfer correlations for natural convection boiling, *Int. J. Heat Mass Transfer* **23**, 73–87 (1980).
101. S. S. Kutateladze, *Fundamentals of the Heat Transfer Theory*, 5th Edn. Atomizdat, Moscow (1979).
102. N. G. Rassokhin, N. K. Shvetsov and A. V. Kuzmin, Calculations of boiling heat transfer, *Thermal Engng* No. 9, 86–90 (1970).
103. G. Guglielmini, E. Nannei and C. Pisoni, Survey of heat transfer correlations in forced convection boiling, *Wärme- und Stoffübertr.* **13**, 177–185 (1980).
104. V. E. Schrock and L. M. Grossman, Forced convection boiling in tubes, *Nucl. Sci. Engng* **12**, 474–481 (1962).
105. R. L. Sani, Downward boiling and non-boiling heat transfer in a uniformly heated tube, UVCRL-9023 (1960).
106. R. W. Bjorge, G. R. Hall and W. M. Rohsenow, Correlation of forced convection boiling heat transfer data, *Int. J. Heat Mass Transfer* **25**, 753–757 (1982).

## UNE FORMULE GENERALE POUR LE TRANSFERT THERMIQUE EN CONVECTION FORCEE DIPHASIQUE—SECONDE DETERMINATION

**Résumé**—Une formule générale pour la convection forcée thermique diphasique (ébullition nucléée et vaporisation) est suggérée pour des canaux verticaux et horizontaux avec un périmètre complètement mouillé. Elle représente les données expérimentales, avec une déviation moyenne égale à 14,4% pour 21 liquides différents (eau, liquides organiques, Fréons et cryogènes) dans les domaines suivants des paramètres principaux: pression 0,61–196 bar, densité de flux thermique  $10\text{--}8 \times 10^6 \text{ W m}^{-2}$ , débit masse 5,6–6240  $\text{kg m}^{-2} \text{ s}^{-1}$ , diamètre du canal 0,47–74,7 mm. Le flux de chaleur transféré par l'ébullition nucléée comme par vaporisation dépend de la conductivité thermique de la paroi du canal. On montre que le transfert de chaleur par ébullition nucléée est décrit par une seule équation en variables dimensionnelles contenant une constante individuelle pour chacun des quatre groupes de fluides (eau, liquides organiques, Fréons et cryogènes). Le transfert de chaleur par vaporisation est décrit par une seule équation avec une constante universelle pour tous les fluides.

## EINE VERALLGEMEINERTE KORRELATION FÜR DEN WÄRMEÜBERGANG BEI ERZWUNGENER ZWEIPHASENSTRÖMUNG—ZWEITE FORMULIERUNG

**Zusammenfassung**—Es wird eine verallgemeinerte Korrelation für den Wärmeübergang bei erzwungener Zweiphasenströmung (Blasensieden und Verdampfung) vorgeschlagen, die in vertikalen und horizontalen Kanälen bei vollständig benetztem Umfang gültig ist. Sie gibt die Meßergebnisse für 21 verschiedene Flüssigkeiten (Wasser, organische Flüssigkeiten, Kältemittel und kryogene Flüssigkeiten) mit einer mittleren absoluten Abweichung von 14,4% und folgendem Bereich der Hauptparameter wieder: Druck, 0,61–196 bar; Wärmestromdichte,  $10\text{--}8 \times 10^6 \text{ W m}^{-2}$ ; Massenstromdichte 5,6–6240  $\text{kg m}^{-2} \text{ s}^{-1}$ ; Kanal-durchmesser, 0,47–74,7 mm. Es zeigt sich, daß der Wärmeübergang sowohl beim Blasensieden als auch bei der Verdampfung von der Wärmeleitfähigkeit des Wandmaterials abhängt. Der Wärmeübergang beim Blasensieden wird mit einer einzigen Gleichung mit einer individuellen Konstanten für jede der vier Flüssigkeitsgruppen dargestellt. Der Wärmeübergang bei der Verdampfung läßt sich mit einer einzigen Gleichung mit einer universellen Konstanten für alle Fluide darstellen.

## ОБОБЩЕННОЕ СООТНОШЕНИЕ ДЛЯ РАСЧЕТА ТЕПЛОТДАЧИ К ДВУХФАЗНОМУ ПОТОКУ ПРИ ВЫНУЖДЕННОЙ КОНВЕКЦИИ—ПОВТОРНАЯ ОЦЕНКА

**Аннотация**—Получено обобщенное соотношение для расчета теплоотдачи при вынужденном движении двухфазного потока (пузырьковом кипении и испарении), справедливое для вертикальных и горизонтальных каналов с полностью смоченным периметром. Соотношение со средним абсолютным отклонением в 14,4% описывает экспериментальные данные для двадцати одной различной жидкости (вода, органические жидкости, фреоны, криогены) в следующем диапазоне основных параметров: давление 0,61–196 бар, плотность теплового потока  $10\text{--}8 \times 10^6 \text{ Вт м}^{-2}$ , массовая скорость 5,6–6240  $\text{кг м}^{-2} \text{ с}^{-1}$ , диаметр канала 0,47–74,7 мм. Установлено, что интенсивность теплоотдачи как при пузырьковом кипении, так и при испарении зависит от теплопроводности материала стенки канала. Показано, что теплоотдача при пузырьковом кипении описывается единым уравнением в безразмерных переменных, содержащим индивидуальную для каждой из четырех групп жидкостей (вода, органические жидкости, фреоны, криогены) константу. Теплоотдача при испарении описывается единым уравнением с константой, универсальной для всех жидкостей.